

Sorption of radionuclides in geologic systems

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This report concerns a study which was conducted for SKBF/KBS. The conclusions and viewpoints presented in the report are those of the author(s) and do not necessarily coincide with those of the client.

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SORPTION OF RADIONUCLIDES IN GEOLOGIC SYSTEMS

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SUMMARY

The sorption of radionuclides (cesium, strontium, iodide, technetium and carbon-14) has been studied as a function of pH, nuclide concentration, ionic strength and contact time on geologic material in distribution experiments (batch-wise equilibrium studies and transport studies in columns).

Important parameters governing the interaction of radionuclides with geologic materials are pH, the ionic strength and the amount of complexforming agents in the aqueous phase, and the mineralogy of the geologic media, the cation exchange capacity of the corresponding minerals, the surface to mass ratio and the amount of complex-forming agents in the solid phase.

Three main sorption mechanisms have been exemplified: <u>Electrostatic</u> <u>interactions</u> (ion exchange processes), which depend on the exchange capacity of the solid, the pH and ionic strength of the water and concentration of the sorbing species; <u>chemisorption processes</u>, where complexforming agents, either in solution or on the solid phase interact with the radionuclide; <u>physical adsorption phenomena</u>, where cationic and neutral complexes are sorbed on solid surfaces by non-specific forces.

The sorption mechanism for cesium and strontium is mainly ion exchange and for iodide, technetium and carbon (as carbonate) chemisorption under oxic conditions. Technetium in the reduced form (as technetium(IV)hydroxide/oxid sorbs like the actinides mainly by physical adsorption.

Radionuclides sorbing by ion exchange mechanisms are found to migrate in rock through microfissures and high-capacity minerals. A similar migration was not found for radionuclides sorbing mainly by chemisorption or physical adsorption processes.

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1. INTRODUCTION

The release of radionuclides into the biosphere from a deep underground repository for high-level radioactive waste or spent fuel will be prevented by both engineered barriers (sparingly soluble waste-form, over-pack, encapsulation, possibly a back-fill with certain additives) as well as natural barriers (low groundwater flow, retention of the nuclides due to the interaction with geologic media). A thorough knowledge of the chemical properties of the various long-lived radionuclides in spent fuel as well as of the interactions with components of the bedrock would be required and a necessary basis for predictions of nuclide mobilities and transport times.

The actinides with daughter products as well as a few long-lived fission products (e.g. 99 Tc, 129 I, 135 Cs; in the short time-perspective 90 Sr and 137 Cs) dominate the biological hazards of spent fuel. A few additional activation products would be present in low- and medium-level waste (e.g. $^{3}_{H}$, $^{13}_{C}$, $^{59}_{Ni}$, $^{95}_{Nb}$).

Some qualitative and quantitative observations of sorption phenomena involving some of the important fission products (primarily Sr, Cs and I; also some data for Tc and carbonate) are discussed in this report, which is largely a summary of previously published or presented material (1-14), representing studies over the period 1978-1982 at the Dept. of Nuclear Chemistry, Chalmers University of Technology, Sweden. Similar summaries of actinide sorption studies have previously been presented (15-16). Comparisons with literature data are given elsewhere (17).

2. THE GROUNDWATER-ROCK SYSTEM

Spent nuclear fuel or high-level reprocessing waste will be stored in crystalline rock, probably of granitic or granodioritic composition, at great depth in the present Swedish concept for radioactive waste deposition. The variety in chemical environments encountered in the proposed bedrock-groundwater systems must be considered in any systematic study of radionuclide behaviour and mobility.

2.1. Mineralogic composition of igneous rocks

Igneous rocks like granite are composed of a small number of major rockforming minerals like quartz, feldspars (orthoclase, microcline, plagioclase), micas (biotite), amphiboles (hornblende), pyroxenes and olivine. Moreover, there are accessory minerals like fluorite, calcite, apatite, magnetite etc. that would have some influence on the interactions with trace elements in the groundwater, especially if these minerals have chemisorbing capacities or particularly high exchange capacities.

Of special interest are the fracture filling products which usually form a coating on water-bearing fractures $(\underline{18},\underline{19})$. Thus, the dissolved trace components in the groundwater will primarily interact with these minerals rather than with the underlying bulk rock components.

2.2. Groundwater composition in igneous rocks

The most important chemical system in many shallow or deep groundwaters is the H_2O-CO_2 -system, which largely controls pH (through the protolysis of H_2CO_3). The total carbonate concentration determines the maximum calcium concentration (precipitation of calcite, $CaCO_3(s)$), which in turn sets an upper limit for the fluoride concentration due to the low solubility of fluorite ($CaF_2(s)$). Some interrelations exist between the calcium, magnesium, sodium and potassium concentrations through ion exchange reactions with weathering products (mostly clay minerals). Thus, the concentrations of HCO_3^{-} , Ca^{2+} , Na^+ , Mg^{2+} , K^+ , F^- (as well as SiO_2 , HPO_4^{-2-} and occasionally SO_4^{-2-}) are interdependent and related to the various solid phases present. Another important interdependent system is $Fe(II)-Fe(III)-O_2$, which largely governs the redox conditions.

Saline waters with total salt concentrations (mainly Na⁺, Ca²⁺ and C1⁻) of up to several thousand mg/1 (normally less than 300 mg/1) are not uncommon in deep groundwaters. (Further aspects on groundwater chemistry is given in ref. 20).

Concentration ranges of major species in deep groundwaters are summarized in Table 1, as well as maximum concentrations in saline waters and the composition of an artificial groundwater previously used in sorption studies (7,15).

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	Normal concentration	Max. concentration b	Artificial
	range	in saline waters	groundwater
HCO ₃	90-275	(400)	123
so ₄ ² -	0.5-15	40(370)	9.6
нро ₄ 2-	0.01-0.2		
F	0.5-5	5(7.5)	
C1 ⁻	5-15	350(6300)	70
Ca ²⁺	10-40	90(2100)	18
Mg ²⁺	1-10	20(210)	4.3
Na	10-100	300(1660)	65
к+	1-5	10(37)	3.9
Fe(II)	0.02-5		
SiO ₂ (tot)	3-14	20	11
TOC ^a	1-8		
рН	7-9		8.2

Table 1 Composition of deep groundwaters (mg/1)

a Total organic carbon

Ъ

Common upper concentration levels are given; values within parentheses represent the highest values encountered in any location.

2.3. Selection of solid and aqueous phases for sorption studies

The general principle in the selection of minerals for the experimental studies has been to include representatives of the most common rockforming components as well as minerals that for various reasons would be expected to interact strongly with the radionuclide (high exchange capacity, chemisorption). The following groups of rocks and minerals are represented:

- o Major rock-forming minerals
- Common accessory minerals, including some fracture filling products and weathering products
- High-capacity minerals (high anion- or cation exchange capacity; see ref. 21)

o Chemisorbing minerals (for I^{-} and $H^{14}CO_{3}^{-}$)

 Hydroxides, representing corrosion products and fresh metal precipatates with large surface/volume ratios o Igneous rocks; granite, gneiss and diabase from Finnsjön, Stripa, Studsvik and Blekinge.

The selected mineral systems are listed in Table 2.

The artificial groundwater, Table 1, representative of deep non-saline groundwaters, was normally used as aqueous phase, and 4 M NaCl was representing very saline waters or brine. The transport studies simulating the field measurements in Studsvik were performed with an artificial groundwater with a composition based on analyses from drilling holes in the area $(\underline{12},\underline{14})$.

Class	Mineral ^a	Mineral ^a	
Sulfides	Galena	Galena	
	Chalcopyrite		I
	Cinnabar		I
	Pyrite		Sr, Tc, Cs
Oxides,	Corundum		Cs
Hydroxides	Hematite		Sr, Cs
	Magnetite		Sr, Tc, Cs
	Limonite		1
	Fe(OH)		Sr, I, Cs
	A1(OH)		Sr, I, Cs
	$Pb(OH)_2$		Sr, 1, Cs
	$Cu(OH)_2$		Sr, Cs
Halides,	Fluorite		Sr, Cs
Carbonates,	Calcite		C, Sr, Cs
Sulfates,	Dolomite		Sr, Us
Phosphates	Gypsum		Sr, US
	Apatite		Sr, US
	Agoi		JI, 1, US
Silicates			
Nesosilicates	Olivine		I
Sorosilicates	Epidote	Epidote	
T (1)	Prehnite		Sr, Cs
Inosilicates	Hornblende		Sr, Tc, Cs
	Attapulgite		Ţ
Phyllosilicates	Kaolinite		Sr, Cs
	Serpentine		Sr,I, Cs
	Halloysite		I
	Illite		Sr, Cs
	Montmorillonite		C, Sr, Cs
	Muscovite		Sr, Cs
	Biotite		Sr, Tc, Cs
	Chlorite		Sr, Tc, Cs
Tectosilicates	Quartz		Sr, Cs
	Microcline		Sr, Cs
	Orthoclase		Sr, Cs
	Plagioclase		Sr, Cs
	Laumontite		Sr, Cs
	Stilbite		Sr, Cs
Igneous rocks	Granite, Finnsjön	(Fi)	Sr, Cs
	Granite, Stripa	(Str)	C, Sr, Tc, Cs
	Gneiss/Granite,		
	Studsvik	(Stu)	Sr, Cs
	Gneiss/Granite,	. .	
	Blekinge	(G-B1)	Sr, Cs
	Diabase, Blekinge	(D-B1)	Sr, Cs

Table 2 Minerals and rocks selected for sorption studies

3. EXPERIMENTAL

3.1. Preparation and characterization of sorbents

Pure, crystalline products from natural sources were used. Mineral grains that appeared to be reasonably pure were selected from a crushed or coarsely ground material and then further ground and sieved. Usually the size fraction 0.045-0.063 or 0.090-0.125 mm was used. Large samples of homogeneous rocks were crushed and sieved, and only the larger size fraction 0.090-0.125 mm was used. The purity was checked by X-ray diffractometry, and for some of the minerals also by chemical analysis. Anion exchange capacities (AEC) and cation exchange capacities (CEC) as well as surface/mass ratios were determined (21). (Detailed data concerning mineral composition and purity, sources etc. are available but not included in this report.) The mineralogic composition of the rock samples is given in Table 3.

Mineral	C	Composition, wt-%	a
	Finnsjön	Stripa	Studsvik
Quartz	39	37	40-70
Plagioclase	26	29	10-40
Microcline	17	26	
Biotite	6	2	b
Muscovite	-	4	
Hornblende	8	-	Ъ
Epidote	3	-	
Accessories	1	2	Ъ

Table 3 Composition of rock samples

^a From ref. 12, 18 and 19

 $^{\rm b}$ Hornblende, biotite and accessories all together 20 weight %

3.2. Sorption studies

3.2.1. Radionuclides

The radionuclides used in the sorption and migration studies are listed in

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Table 4. Acidic stock solutions (pH 1-2) with a total concentration of $10^{-5} - 10^{-6}$ M were generally prepared.

3.2.2. Batch measurements

The crushed (in agate mortar) and sieved material was prewashed and equilibrated with aqueous phase for one week. The solid was allowed to settle by gravity or by a light centrifugation after a short vigorous shaking of the sample. The aqueous phase was removed and discharged and new aqueous phase was added. The system was allowed to equilibrate for another week before the acidic active nuclide solution was added (typically 0.1 ml). After the addition of the spike solution pH was adjusted to roughly the desired level (between 5 and 10) by the use of 0.1 or 0.01 M NaOH or HC1.

The activity remaining in the system was determined using a Ge(Li)-detector (after phase separation by centrifugation), as well as pH, after various contact times. The distribution coefficient $K_d (m^3/kg)$ was calculated according to

$$K_{d} = (c_{o} - c)V/cm$$
(1)

where c_{0} = initial concentration (M),

c $\,$ = measured concentration after a certain contact time (M) $\,$

V = volume of aqueous phase (m³)

m = mass of solid phase (kg).

The sorption on the walls of the vials was measured and found to be negligible in the presence of crushed material with a large accessible surface.

The composition of the artificial groundwater was usually not significantly altered under the present experimental conditions except for pH $(\underline{15})$.

All experiments were performed in aerated systems, except for some batch measurements, where the technetium sorption under reducing conditions was studied.

The experimental conditions are given in Table 5.

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Nuclide	Half-life	
¹⁴ c ^a ⁸⁵ Sr ^{99m} Tc ^b	5730 y 64.8 d 6.0 h	
¹³¹ I ¹³⁴ Cs	8.0 d 2.06 y	

Table 4 Radionuclides used in the sorption studies.

^a As H¹⁴CO₃ ^b From ⁹⁹Mo, $t_{\frac{1}{2}} = 66$ h

Table 5 Conditions for batch experiments

Solid phase: Grain size	Minerals according to Table 2	0.045-0.063 mm 0.090-0.125 mm
Aqueous phase:	Artificial groundwater according Various salt solutions	to Table l (<u>standard</u>)
Solid/liquid Initial eleme	ratio ent concentration	ca 0.5-lg per 45 ml ^a (10-20 g/l) usually 10 ⁻⁷ -10 ⁻⁸ M ^a (standard)
Other parameters		22+2 ⁰ C
Contact time		ld, lw (<u>standard</u>) or longer
Phase separat	ion	Centrifugation; lh at 4000g (27000g for some clay systems)
Equipment		Polypropylene, glass

- а
- Exact amounts of solids and aqueous phases are known for each of the ca 1500 measurements given in the subsequent figures and table but not included in this report.

3.2.3. Column measurements

A column, 0.1 m high and \emptyset 4 mm, was packed with crushed rock or mineral (0.090-0.125 mm), and a pulse of radionuclide solution (usually 0.05 ml) was added to the column. Artificial groundwater was pumped through the column by means of a HPLC-pump and was collected by an automatic fraction collector.

The flow rate through the column was varied between 0.01 and 0.08 ml/min.

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The retention of a certain radionuclide in the column was obtained from comparison with a non-sorbing tracer. The distribution coefficient was estimated from:

$$R = u_w / u_p = 1 + K_d \rho_p (1 - \varepsilon) / \varepsilon$$
⁽²⁾

where R = retention factor u_n = velocity of nuclide (m/s) u_n = velocity of water (= velocity of non-sorbing tracer) (m/s) ρ_p = density of solid (m³/kg) ϵ = porosity of column filling.

3.2.4. In situ measurements

Boreholes in the bedrock at the Studsvik laboratory area were used for retention studies <u>in situ</u>. (12,14) The mineralogy of a drilling core from the area was determined and fracture filling minerals identified. Two boreholes with an interconnecting fracture at about 100 m depth were chosen for the migration experiments. A pump was installed in one of the holes to remove and sample water coming from the fracture. The composition of this water was determined at regular intervals during the preparations and subsequent experiments. The hydraulic properties of the flow paths were determined by adding a pulse of ¹³¹I to the injection hole followed by **Y**-activity measurements of the water coming from the pump-hole. The result in terms of water flow conditions was interpreted as a combination of several flowpaths where one with a residence time of 288 h was dominating. After this preliminary test-run a simultaneous injection of ³H, ⁸⁵Sr and ¹³¹I was performed and the activity in the pumphole water was recorded vs. time.

4. RESULTS AND DISCUSSION

4.1. Strontium and cesium

4.1.1. Composition of the solid

The sorption, expressed as the distribution coefficient, of cesium and strontium on minerals and rocks is given in Figure 1 and 2, respectively and in Table 6a-c (7,10,11).

Cesium does not form complexes with species normally found in groundwaters, and interacts with geologic materials as the non-complexed monovalent cation Cs⁺. The sorption is mainly a cation exchange process dependent both on the pH of the water and on the properties of the solid. As expected for cation exchange processes the sorption of cesium increases with increasing pH of the water (Figure 1). Furthermore, minerals with high CEC, like biotite and hornblende exhibit high cesium sorption, while minerals with low CEC (e.g. quartz) sorb cesium poorly (cf. Figure 3).

Cesium was found to sorb on natural fracture surfaces, especially on fracture fillings with high CEC with a consecutive diffusion through the fracture filling into the bulk rock. Cesium migrated millimeters into the rock at an exposure time of a couple of months of the fracture surface to a spiked groundwater. The major transport pathways into the rock were found to be microfissures and high-capacity mineral veins (13).

The sorption of strontium is, similar to cesium, largely a cation exchange process; that is, highly related to the pH of the water and the CEC of the solid (cf. Figure 2 and 3). Strontium acts predominantly as the divalent cation Sr^{2+} in most environmental waters. At high pH (>9) and fairly high carbonate concentration, a formation of hydrolysis products or carbonate complexes is possible leading to an increased sorption due to a physical adsorption of neutral or near neutral complexes on the solid.

High sorption of strontium was found in phosphate or fluoride rich waters, probably due to a formation of sparingly soluble strontium phosphate and fluoride complexes.

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	K _d , m ³ /kg				
Solid	рH	Sr	Cs		
 Pyrite	8-8.5	0.003	0.003		
Corundum	7-8		0.000		
Hematite	4-6	0.003	0.002		
	6-8	0.004	0.005		
	8-9	0.007	0.006		
Magnetite	4-6	0.002			
	6-8	0.002	0.007		
	8-10	0.002	0.007		
Fluorite	4-6	0.003	0.000		
	6-8	0.003	0.000		
	8-9	0.002	0.000		
Calcite	6-8	0.002			
	8-9	0.003	0.001		
a	8-8.5	0.004	0.006		
Dolomite	8-8.5	0.003	0.005		
Gypsum	8-8.5	0.003	0.006		
Apatite	4-6	0.003	0.006		
-	6-8	0.005	0.000		
	8-9	0.008	0.003		
Epidote	8-8.5	0.010	0.019		
Prehnite	8-8.5	0.010	0.013		
b	8-8.5	0.016	0.048		
Hornblende	4-6	0.002			
	6-8	0.005	0.156		
	8-9	0.018	0.213		
Biotite	4-6	0.004	0.925		
	6-8	0.016	0.610		
	8-9	0.024	0.624		
Muscovite	4-6	0.014	0.684		
	6-8	0.071	0.591		
	8-9	0.114	0.765		
Serpentine	8-9	0.010	0.012		
Chlorite	8-8.5	0.050	0.128		
Kaolinite	8-8.5	0.029	0.059		
Illite	8-8.5	0.335	> 5		
Bentonite ^C	8-9	3.5	1.1		
Quartz	4-6	0.000			
	6-8	0.000			
	8-9	0.001	0.001		
Orthoclase	4-6	0.001	0.007		
	6-8	0.002	0.013		
	8-9	0.005	0.011		
Plagioclase	8-8.5	0.004	0.011		
Microcline	8-8.5	0.012	0.177		
	0 0 5	0 0 0 0	0 117		
Laumonite	88.5	0.029	0.117		

Table 6a Distribution coefficients for cesium and strontium on minerals from batch experiments. (Table 5, groundwater systems)

a Fracture filling in drill core at 309 m depth, Finnsjön

b Fracture filling in drill hole at 519 m depth, Finnsjön

^c Commercial Wyoming bentonite, MX-80, mainly Na-montmorillonite

	K_d , m^3/kg				
Solid	рН	Sr	Cs		
Stripa, granite	3-7		0.089		
	7-9	0.006	0.152		
	9.11	0.057	0.152		
Finnsjön, granite	3-7	0.007	0.264		
	7-9	0.020	0.490		
	9-11	0.259	0.639		
Studsvik,	6-7	0.008	1.215		
gneiss/granit	7-8	0.012	0.106		
	8-9	0.021	0.948		
a	7-8	0.004			
а	8-9	0.006			
а	> 9	0.030			
b	7-8	0.043	1.214		
Blekinge, gneiss	3-7	0.001	0.100		
	7-9	0.009	0.107		
	9-11	0.103	0.083		
Blekinge, diabase	3-7	0.006	0.116		
	7-9	0.010	0.368		
	9-11	0.163			

Table 6b Distribution coefficient for cesium and strontium on rocks from batch experiments. (Table 5, groundwater systems)

a In natural water from Studsvik borehole

^b Artificially weathered, autoclave 150[°]C, 1 week

Table 6c Distribution coefficients for cesium and strontium on hydroxides (corrosion products) from batch experiments. (Table 5, groundwater systems)

		K_d , m^3/kg		
Solid	рН	Sr	Cs	
Fe(OH) ₃	8-8.5	1.20	0.010	
Pb(OH) ₂	8-8.5	0.000	0.000	
Cu(OH) ₂	8-8.5	0.001	0.000	





- 5, groundwater systems); Effect of pH
- a. Non-silicates
 - Magnetite, □ Hematite, Calcite,
 - Apatite
- b. Silicates
 - Hornblende, Muscovite, □ Biotite,
 - Orthoclase, \\$ Quartz
- c. Rocks
 - Granite-Fi, Granite-Str, 🗆 Gneiss-Stu



<u>Figure 1</u> Distribution coefficients for cesium from batch experiments; Effect of pH (continued)

- c. Rocks
 - Granite-Fi, Granite-Str, □ Gneiss-Stu



Figure 2 Distribution coefficients for strontium from batch experiments (Table 5, groundwater systems); Effects of pH

- a. Non-silicates
 - Magnetite, □ Hematite, Calcite,
 - 🖬 Apatite, 🌣 Fluorite
- b. Silicates
 - Hornblende, Muscovite, □ Biotite,
 - 📕 Orthoclase, 🌣 Quartz
- c. Rocks
 - Granite-Fi, Granite-Str, □ Gneiss-Stu



<u>Figure 2</u> Distribution coefficients for strontium from batch experiments; Effect of pH (continued)

c. Rocks

● Granite-Fi, ○ Granite-Str, 🗌 Gneiss-Stu



Figure 3 Distribution coefficients for cesium and strontium at pH 8; Effect of cation exchange capacity (CEC) of the solid Minerals and rocks in order of increasing CEC: Calcite, magnetite apatite, hematite, orthoclase, gneiss-Stu, diabase, serpentine, hornblende, granite-Str, granite-Fi, biotite, muscovite.

4.1.2. Nuclide concentration and ionic strength

The ionic strength is important for ion exchange processes, there is a stronger competition for the sorption sites on the solid in a concentrated ionic media than in a dilute. In Table 7 the distribution coefficients are given for cesium and strontium in three ionic medias; groundwater, 0.1 M NaCl and 4 M NaCl. As expected the distribution coefficients are much lower in the 4 M salt solution than in groundwater and 0.1 M NaCl (cf. Figure 4) (10). For low nuclide concentrations ($<10^{-7}$ M) on solids with high CEC (e.g. granite, hornblende, montmorillonite, etc.) linear sorption isotherms have been found. In these cases the sorption follows a Freundlich type of sorption isotherm, expressed as

 $q = Kc^n$

where q = concentration in solid (mol/kg), c = concentration in liquid (mol/m³) and K and n are system constants (K is the distribution coeffici-

ent K_d). For bentonite n is found to be 0.94 and 0.91 for cesium and strontium, respectively. Different exponents have been calculated for the granites, e.g. for cesium 0.78 on Studsvik granite and 0.57 on Stripa granite.

		K _d , m ³ ,	/kg
Solid	Ionic medium	Cs	Sr
Fi	Groundwater	0.331	0.014
	0.1 M NaCl	0.135	0.016
	4 M NaCl	0.005	0
Str	Groundwater	0.043	0.006
	0.1 M NaCl	0.160	0.007
	4 M NaCl	0.015	0
G-B1	Groundwater	0.138	0.003
	0.1 M NaCl	0.065	0.004
	4 M NaCl	0.002	0
D-Be	Groundwater	0.220	0.005
	0.1 M NaC1	0.100	0.008
	4 M NaCl	0.004	0

<u>Table 7</u> Influence of ionic strength on the distribution of cesium and strontium



Figure 4 Sorption isotherms for cesium and strontium.

q = concentration in solid, c = concentration in solution
a. cesium

- Hornblende, Orthoclase

4.1.3. Contact time

The distribution coefficients for cesium and strontium vs. contact time are illustrated in Figure 5. Two different sorption mechanisms are found for both cesium and strontium; first an initial fast (minutes or hours) surface dependent reaction followed by a slower (up to years) process, probably related to a diffusion into the solid matrix. After long contact time (more than 6 months) the weathering and alteration of the solid could lead to either an increased sorption or a desorption of the nuclides (10).



<u>Figure 5</u> Distribution coefficients for cesium and strontium; Effect of contact time

a. Silicates

 \Box Hornblende, \bigcirc Orthoclase, \rightleftarrows Biotite

- b. Rocks
 - O Fi, □ Stu, ≯ Str

cesium (filled symbols) and strontium (open symbols)

4.2. Iodide

Iodine in groundwater will be present mainly as the iodide ion, I, but other species may be formed under influence of light or bacteria (23,24). Since most minerals have a negative surface charge in groundwater environment the anion exchange capacity (AEC) is low compared to CEC.

The sorption of iodide on minerals is given in Table 8 (23) and Figures 6 and 7. Iodides of low solubility are formed by a number of metals (Hg, Ag, Bi, Cu, Pb, etc), thus minerals containing these could sorb iodide by chemisorption. In Table 9 the solubility products of some metal iodides are given and when comparing these with the data in Table 9, it is worth noting that cinnabar contains Hg, chalcopyrite Cu and galena Pb, i.e. K_d increases with decreasing solubility of the iodides. At high ionic strength the sorption decreases even for these minerals (cf. Figure 7). There are some other minerals that are sorbing iodide, probably due to large surface areas. The highest sorption is obtained at low pH as expected for an anion exchange process.

For Stripa granite no sorption has been detected while a low but significant K_d (0.004 m^3/kg) was observed for Studsvik granite at 1 week contact time.

		K ₄ , m ³ /	K _d , m ³ /kg		
Solid	рH	Artificial groundwater	4 M NaCl		
Chalcopyrite	4-6	0.125	0.005		
	6-8 8-10	0.025	0.003		
	0-10	0.003	0.001		
Galena	2-4	0.040	0.002		
	4-6	0.012	0.002		
	6-8 9 10	0.006	0.001		
	8-10	0.004	0.002		
Cinnabar	2-4	1.836	0.030		
	4-6	-	0.020		
	6-8	0.187	0.025		
	8-10	0.043	0.011		
Limonite	4-6	0.019	0.037		
	6-8	0.010	0.008		
	8-10	0.000	0.003		
Olivine	4-6	0.066	0.012		
	6-8	0.005	0.006		
	8-10	0.001	0.003		
Attapulgite	4-6	0.000	0.005		
	6-8	0.048	0.012		
	8-10	0.072	0.008		
Serpentine	4-6	0.022	0.084		
-	6-8	0.011	0.008		
	8-10	0.008	0.003		
Montmorillonite	2-4	0.010	0.151		
	4-6	0.007	0.071		
	6-8	0.000	0.050		
	8-10	0.000	0.005		
Hallovsite	2-4	0.020	-		
2	4-6	_	0.054		
	6-8	0.005	0.015		
	8-10	0.005	0.015		
Quartz	2-4	0.008	0.005		
•	4-6	0.001	0.000		
	6-8	0.001	0.000		
	8-10	0.000	0.000		

Table 8 Distribution coefficients for iodide from batch experiments (Table 5)

Metal	log K s		
Cu ⁺	-12.0	 	
Ag ⁺	-16.08		
Hg ²⁺	-28.33		
РЪ ²⁺	- 8.10		

<u>Table 9</u> Solubility products for metal iodides (25)



Figure 6 Distribution coefficients for iodide from batch experiments (Table 5, groundwater)

- a. Hydroxides
 - Limonite, Fe(III)-hydroxide
 - 🗌 Al-hydroxide, 📓 Pb(II)-hydroxide
- b. Silicates
 - Olivine, M Attapulgite, Serpentine,
 - ¥ Halloysite, □ Montmorillonite, ≯ Quartz
- c. Others
 - Chalcopyrite, Galena, □ Cinnabar, AgC1



<u>Figure 6</u> Distribution coefficients for iodide from batch experiments (continued)

c. Others

○ Chalcopyrite, ● Galena, □ Cinnabar, ■ AgCl



Figure 7 Distribution coefficients for iodide from batch experiments (Table 5, 4 M NaCl)

- a. Hydroxides
 - Limonite, Fe(III)-hydroxide
 - 🗌 Al-hydroxide, 📓 Pb(II)-hydroxide
- b. Silicates
 - 🔿 Olivine, 📓 Attapulgite, 🎱 Serpentine,
 - ¥ Halloysite, □ Montmorillonite, ≯ Quartz
- c. Others
 - 🔿 Chalcopyrite, 🌒 Galena, 🗌 Cinnabar, 📓 AgCl





c. Others

○ Chalcopyrite, ● Galena, □ Cinnabar, ■ AgCl

4.3. Technetium

Technetium exists predominantly as pertechnetate, TcO_4^- , in aqueous solution under oxidizing conditions, and in the tetravalent state, either as a hydroxide or as an oxide, in a reducing environment (6).

The sorption of pertechnetate is poor on geologic materials (cf. Table 10), but in the presence of a reducing agent (Fe(II)) there is a small but significant sorption, probably an adsorption or precipitation of technetium (IV)-hydroxide/oxide (<u>3</u>).

Table 10 Distribution coefficients for technetium (TcO₄) from batch experiments (Table 5, groundwater system, ld contact time)

Solid	Aqueous phase	K _d , m ³ /kg		
Granite	Aereated	< 0.001		
	Deaerated	< 0.001		
	Deaerated, 20 ppm Fe(II)	0.050		
Biotite				
Chlorite				
Hornblende	Deaerated	0.001-0.003		
Pyrite				
Magnetite	Deaerated	0.006		

4.4. Carbonate

Groundwater contains a fairly high concentration of carbonate/hydrogen carbonate (in the mM-range), partly from exchange and contact with the biosphere and the atmospere.

Carbon-14 released into the groundwater will participate in the continuous precipitation/dissolution of calcium carbonate (e.g. the formation and precipitation of the fracture mineral calcite) and will thus be retained compared to the groundwater transport.

The sorption of 14 C, as ${\rm H}^{14}$ CO $_3^-$, on calcium and/or carbonate containing minerals, granite and concrete has been determined in batch experiments (cf. Table 11) (5,8). A significant sorption is found for clayish moraine,

calcite and concrete. The high sorption on concrete is probably because of a formation and precipitation of $CaCO_3(s)$.

	$K_d \times 10^3$, m ³ /kg					
Solid	3d	lw	5w	6 months	months	
Stripa granite	0	0	0	0		
Na-montmorillonite	0	0	0	0		
Bentonite/quartz	2.8	8.6	7.6	7.8		
Sandy moraine	0.8	1.1	2.6	2.2		
Clayish moraine	1.3	2.0	3.0	2.3		
Calcite	3.7	3.5	9.0	83		
Concrete	5.3	1600	>104			

Table 11 Distribution coefficients for HCO₃ from batch experiments (Table 5, groundwater)

4.5. Comparison of batch and column measurements

Retention factors (water velocity/nuclide velocity) and distribution coefficients obtained from column measurements are given in Table 12. When comparing these K_d -values with K_d -values from batch measurements it is obvious that at least for Cs, batch data seems to be higher than column data. The only measurement for strontium on granite indicates the opposite, but more measurements would be required to determine this. A deviation towards lower K_d -values in column measurements would, however, be expected when sorption kinetics are slow, as the contact times in the column are much shorter than in batch measurements.

In a natural fissure the water velocity will be 5-6 orders of magnitude lower than in the laboratory column experiments and the sorption kinetics will probably have less influence on the sorption.

Solid	Tracer	Retention	$K_{\rm d} \propto 10^3$, m ³ /kg		
		factor	Column	Batch	
Calcite	Cs	3	0.5	2	
Quartz	Cs	4	0.8	1	
Orthoclase	Cs	11	3	10	
Hematite	Cs	>22	>3	6	
Magnetite	Cs	>100	>12	6	
Hornblende	Cs	>250	>50	190	
Granite, Stu	Sr	89	20	12	

<u>Table 12</u> Comparison of distribution coefficients (K_d) in batch and column experiments

4.6. In situ measurements

A retention factor of the same order of magnitude as for laboratory column tests for crushed bulk rock has been obtained in the field test (R = 15-17 in field and 30-35 at lab) (<u>12</u>). The difference may be explained by the fact that the column study was performed on drill cuttings from the borehole (i.e. representative for the average rock composition) whereas in the field test the radionuclide interact with the fracture surfaces, which may have slightly different sorption properties. As distribution coefficients for Sr do not differ by more than one order of magnitude for most rocks and minerals, the expected differences between field and laboratory measurements due to different mineral composition are of the same order of magnitude as that measured.

In the experiment a small fraction of strontium was transported with the velocity of the water. This may be due to some transport of strontium sorbed on particles in the water (i.e. FeOOH(s) which may precipitate when the water is areated during the injection), an effect that has been observed also in column studies of radionuclides in clay suspensions (pseudocolloids).

4.7. Predictions of distribution coefficients

There are many different parameters determining the distribution coefficient. One parameter that will vary in nature is the mineralogic composition of the rock, and it would be of great interest to be able to predict the effect of a change in mineralogy without performing laboratory measurements for each rock. A comparison of predicted K_d -values for granites, using data for the pure minerals and measured K_d -values has been performed for cesium, strontium, americium and neptunium, cf. Table 13. For cesium and strontium the predicted values are too low, while the difference for the actinides is smaller. Not even the probable presence of small amounts of weathering products with high K_d -values can wholly explain the deviations for cesium and strontium. For actinides the mineral composition would be expected to be of minor importance as K_d -values usually are within one order of magnitude at the same pH.

Table 13	Comparison	of	predicted	and	measured	distribution	coefficients

Rock	Predicted K _d , m ³ /kg				Me	Measured K_d , m^3/kg		
	Cs	Sr	Am	Np	Cs	Sr	Am	Np
Granite, Str	0.056	0.002		-	0.15	0.015	_	-
Granite, Fi	0.062	0.002	-	-	0.49	0.03	-	-
Climax stock granite ^a	-	-	8	0.050	_	_	20	0.100
Westerly granite ^a		-	6.3	0.050	time.	-	2.5	0.032

^a See ref <u>15</u>

5. CONCLUSIONS

The sorption of radionuclides at trace concentrations in a geologic groundwater environment is related to both physical and chemical parameters. Physical parameters of importance are

- o the groundwater velocity (determining the contact time)
- o the surface/mass ratio of the solid
- o the surface texture and frequency of microfissures in the solid.

The most important chemical parameters of the aqueous phase are

o pH

- o the redox potential (determining the oxidation state)
- o presence of complex-forming agents,

and of the solid phase

- o the mineralogy and degree of weathering
- o the CEC (cation exchange capacity)
- o presence of reducing agents (e.g. divalent iron).

Three main categories of sorption mechanisms influences the over-all transport rate of radionuclides. These processes are: <u>Electrostatic</u> <u>interactions</u> (ion exchange processes) related to the exchange capacity of the solid, the pH and ionic strength of the water and the concentration of the sorbing species; <u>chemisorption processes</u>, where complex-forming agents, either in solution or on the solid phase, interact with the radionuclide; <u>physical adsorption phenomena</u>, where cationic or neutral species of mainly oxides/hydroxides interact with solid surfaces by non-specific adsorption forces.

Cesium and strontium which are non-hydrolyzed metals in environmental waters, participate mainly in ion exchange reactions in geologic systems. Hydrolyzed species, like the actinides and tetravalent technetium are expected to interact through adsorption processes and to some extent by chemisorption/mineralization with geologic media. The negatively charged species, iodide and carbonate, have a sorption largely related to the presence of metals forming sparingly soluble iodide and carbonate complexes, thus interacting through chemisorption processes.

The sorption of redox-sensitive nuclides, like technetium, uranium, neptunium and plutonium, is strongly related to the oxidation state, determined by the redox conditions of the system.

Radionuclides participating in ion exchange reactions, like cesium, are expected to sorb strongly on high-capacity minerals on the fracture surface and be transported into the bulk rock in microfissures and highcapacity minerals. A similar migration into the rock is not expected for nuclides sorbing mainly by chemisorption or physical adsorption.

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7. REFERENCES

- B. Allard, J. Rydberg, H. Kipatsi and B. Torstenfelt, "Disposal of Radioactive Waste in Granitic Bedrock", in S. Fried (Ed.), Radioactive Waste in Geologic Storage, ACS Symposium Series 100, American Chem. Soc., Washington D.C. 1979, p. 47.
- B. Allard, H. Kipatsi, B. Torstenfelt and J. Rydberg, "Nuclide Transport by Groundwater in Swedish Bedrock", in G.J. McCarthy (Ed.), Scientific Basis for Nuclear Waste Management. Vol. 1, Plenum Publ. Corp., New York 1979, p. 403.
- B. Allard, H. Kipatsi and B. Torstenfelt, "Technetium: Reduction and Sorption in Granitic Bedrock", Radiochem. Radioanal. Letters <u>37</u> (1979) 223.
- 4. K. Andersson and B. Torstenfelt, "Leaking of ⁹⁴Nb From an Underground Rock Repository", KBS TR 79-26, Kärnbränslesäkerhet, Stockholm 1980.
- 5. B. Allard, B. Torstenfelt and K. Andersson, "Sorption Behaviour of ¹⁴C in Groundwater/rock and in Groundwater/concrete Environment", Report Prav 4.27, Programrådet för radioaktivt avfall, Stockholm 1981.
- 6. B. Torstenfelt, B. Allard, K. Andersson and U. Olofsson, "Technetium in the Geologic Environment - A Literature Survey", Report Prav 4.28, Programrådet för radioaktivt avfall, Stockholm 1981.
- B. Torstenfelt, K. Andersson and B. Allard, "Sorption of Strontium and Cesium on Rocks and Minerals. Part I. Sorption in Groundwater", Report Prav 4.29, Programrådet för radioaktivt avfall, Stockholm 1981.
- B. Allard, B. Torstenfelt and K. Andersson, "Sorption Studies of H¹⁴CO₃" on Some Geologic Media and Concrete", in J.G. Moore (Ed.), Scientific Basis for Nuclear Waste Management. Vol. 3, Plenum Publ. Corp., New York 1981, p. 465.
- B. Allard, S.Å. Larson, Y. Albinsson, E.-L. Tullborg, M. Karlsson,
 K. Andersson and B. Torstenfelt, "Minerals and Precipitates in

Fractures and Their Effects on the Retention of Radionuclides in Crystalline Rocks", Proc. Workshop on Near-Field Phenomena in Geologic Repositories for Radioactive Waste, Seattle 1981, OECD/NEA, Paris 1982, p. 93.

- B. Torstenfelt, K. Andersson and B. Allard, "Sorption of Strontium and Cesium on Rocks and Minerals", Chem. Geol. 36 (1982) 123.
- 11. K. Andersson, B. Torstenfelt and B. Allard, "Sorption Behaviour of Long-lived Radionuclides in Igneous Rocks", in Environmental Migration of Long-lived Radionuclides, IAEA, Vienna 1982, p. 111.
- 12. O. Landström, C.E. Klockars, O. Persson, K. Andersson, B. Torstenfelt, B. Allard, S.Å. Larson and E.-L. Tullborg, "A Comparison of In-situ Radionuclide Migration Studies in the Studsvik Area and Laboratory Measurements", in W. Lutze (Ed.), Scientific Basis for Nuclear Waste Management. Vol. 5, Elsevier Sci. Publ. Comp., New York 1982, p. 697.
- 13. B. Torstenfelt, T. Eliasson, B. Allard, K. Andersson, S. Höglund, T. Ittner and U. Olofsson, "Radionuclide Migration Into Natural Fracture Surfaces of Granitic Rock", in D.G. Brookins (Ed.), Scientific Basis for Nuclear Waste Management. Vol. 6, Elsevier Sci. Publ. Comp., New York 1983.
- 14. O. Landström, C.E. Klockars, O. Persson, K. Andersson, B. Torstenfelt,
 B. Allard, E.-L. Tullborg and S.Å. Larson, "Migration Experiments in Studsvik", KBS TR 83-18, Svensk Kärnbränsleförsörjning AB, Stockholm 1983.
- B. Allard, "Sorption of Actinides in Granitic Rock", KBS TR 82-21, Svensk Kärnbränsleförsörjning AB, Stockholm 1982.
- 16. B. Allard, U. Olofsson, B. Torstenfelt and H. Kipatsi, "Sorption of Actinides in Well-defined Oxidation States", KBS TR 83-61, Svensk Kärnbränsleförsörjning AB, Stockholm 1983.
- K. Andersson and B. Allard, "Sorption of Radionuclides on Geologic Media - A Literature Survey. I. Fission Products", KBS TR 83-07, Svensk Kärnbränsleförsörjning AB, Stockholm 1983.

- 18. S.Å. Larson, E.-L. Tullborg and S. Lindblom, "Sprickmineralogiska undersökningar", Rapport Prav 4.20, Programrådet för radioaktivt avfall, Stockholm 1981.
- E.-L. Tullborg and S.Å. Larson, "Fissure Filling from Finnsjön and Studsvik, Sweden", KBS TR 82-20, Svensk Kärnbränsleförsörjning AB, Stockholm 1982.
- 20. B. Allard, S.Å. Larson, E.-L. Tullborg and P. Wikberg, "Chemistry of Deep Groundwaters from Granitic Bedrock", KBS TR 83-59, Svensk Kärnbränsleförsörjning AB, Stockholm 1983.
- 21. B. Allard, E.-L. Tullborg, S.Å. Larson and M. Karlsson, "Ion Exchange Capacities and Surface Areas of Some Major Components and Common Fracture Filling Materials of Igneous Rocks", KBS TR 83-69, Svensk Kärnbränsleförsörjning AB, Stockholm 1983.
- 22. C.S. Hurlbut, <u>Dana's Manual of Mineralogy</u>, John Wiley & Sons Inc., New York 1966.
- 23. B. Allard, B. Torstenfelt, K. Andersson and J. Rydberg, "Possible Retention of Iodine in the Ground", in C.J.M. Northrup (Ed.), Scientific Basis for Nuclear Waste Management. Vol. 2, Plenum Publ., New York 1981, p. 673.
- 24. H. Behrens, "New Insights Into the Chemical Behaviour of Radioiodine in Aquatic Environments", in Environmental Migration of Long-lived Radionuclides, IAEA, Vienna 1982, p. 27.
- 25. L.G. Sillén and A.E. Martell, "<u>Stability Constants of Metal Ion</u> <u>Complexes</u>, Special Publ. No 17, The Chemical Society, London 1964 and <u>Stability Constants of Metal Ion Complexes</u>, <u>Supplement I</u>, Special Publ. No. 25, The Chemical Society, London 1971.

Appendix Al

SELECTION OF DISTRIBUTION COEFFICIENTS FOR A GRANITE

A list of recommended distribution coefficients for a pilot case used in migration modeling has been compiled, cf. Table Al. The groundwater and rock compositions are typical of the Finnsjön area north of Stockholm and the recommended K_d-values are to be regarded as conservative (i.e. they may be higher but not lower).

Some comments to the choice of constants:

<u>Cobalt and nickel</u> may be expected to have a similar behaviour. There are very few measurements of distribution coefficients for these metals on granite, but a value of 0.4 m³/kg at trace concentration of Ni and pH 8 has been measured at CTH. A recommended value for Co and Ni of 0.2 m³/kg seems feasible, considering a possible higher ionic strength in Finnsjön water than in the synthetic groundwater used in the measurement.

<u>Strontium</u> is sorbed mainly by an ion exchange mechanism and is thus sensitive to ionic strength and pH. The proposed $0.004 \text{ m}^3/\text{kg}$ is reduced due to the risk of high ionic strengths at Finnsjön. In groundwater of lower ionic strength a higher K_d - at least $0.010 \text{ m}^3/\text{kg}$ would be expected for a granite of similar composition. For fracture filling minerals from Finnsjön lower K_d -values than that for the pure granite have been obtained.

Zirconium and niobium have been found to have very high K_d -values at pH 8-9 and here 4 m³/kg may be a conservative value. At higher pH K_d decreases.

<u>Technetium</u> is redox sensitive but even under oxidizing conditions a $K_d \neq 0$, although probably <0.001 m³/kg may be expected. At short contact times (~ld) distribution coefficients of 0.050 m³/kg have been found under reducing conditions, and this may be regarded as a lower limit for longer contact times.

<u>Iodide</u> may be sorbed by some minerals, but there is no reason to use $K_d > 0$ for a safety analysis as granites usually have $K_d \approx 0$.

<u>Cesium</u> sorption is dependent on pH, ionic strength and nuclide concentration. In the case of high ionic strength a quite low value has been chosen - 0.050 m³/kg. Much higher values have, however, been encountered for granites. These values are for trace concentrations ($\sim 10^{-8}$ M) of cesium. At higher concentrations lower K_d may be expected. Cerium, neodymium and europium may be expected to behave very similarly to Th, U and Am and thus undergo hydrolysis. The distribution coefficients will be high; >5 m³/kg may be expected (cf. ref. <u>15</u> where data for all the actinides are discussed).

Nuclide		Granite	Fracture fillings	Comments
Со		0.2		
Ni		0.2		
Sr		0.004	> 0.001	
Zr		4		
Nb		4		
Tc	ox	0.0002		
	red	0.05		
I		0		
Cs		0.05	> 0.001	
Ce, Nd, Eu		<u>></u> 5		
Ra		0.1		
Th		<u>></u> 5		
Pa		<u>></u> 5		
U	ox	0.01		U(VI)
	red	<u>></u> 5		U(IV)
Np	ox	0.01		Np(V)
	red	<u>></u> 5		Np(IV)
Pu	ox	3		Pu(IV)+Pu(V)
	red	<u>></u> 5		Pu(III)
Am		<u>></u> 5		

Table Al Distribution coefficients for pilot case "Finnsjön"

1977-78

TR 121 KBS Technical Reports 1 - 120. Summaries. Stockholm, May 1979.

1979

TR 79-28 The KBS Annual Report 1979. KBS Technical Reports 79-01--79-27. Summaries. Stockholm, March 1980.

1980

TR 80-26 The KBS Annual Report 1980. KBS Technical Reports 80-01--80-25. Summaries. Stockholm, March 1981.

1981

TR 81-17 The KBS Annual Report 1981. KBS Technical Reports 81-01--81-16 Summaries. Stockholm, April 1982.

1983

TR 83-01 Radionuclide transport in a single fissure A laboratory study Trygve E Eriksen Department of Nuclear Chemistry The Royal Institute of Technology Stockholm, Sweden 1983-01-19

TR 83-02 The possible effects of alfa and beta radiolysis on the matrix dissolution of spent nuclear fuel I Grenthe I Puigdomènech J Bruno Department of Inorganic Chemistry Royal Institute of Technology Stockholm, Sweden January 1983

- TR 83-03 Smectite alteration Proceedings of a colloquium at State University of New York at Buffalo, May 26-27, 1982 Compiled by Duwayne M Anderson State University of New York at Buffalo February 15, 1983
- TR 83-04 Stability of bentonite gels in crystalline rock -Physical aspects Roland Pusch Division Soil Mechanics, University of Luleå Luleå, Sweden, 1983-02-20
- TR 83-05 Studies in pitting corrosion on archeological bronzes - Copper Åke Bresle Jozef Saers Birgit Arrhenius Archaeological Research Laboratory University of Stockholm Stockholm, Sweden 1983-01-02

TR 83-06 Investigation of the stress corrosion cracking of pure copper L A Benjamin D Hardie R N Parkins University of Newcastle upon Tyne Department of Metallurgy and Engineering Materials Newcastle upon Tyne, Great Britain, April 1983

- TR 83-07 Sorption of radionuclides on geologic media -A literature survey. I: Fission Products K Andersson B Allard Department of Nuclear Chemistry Chalmers University of Technology Göteborg, Sweden 1983-01-31
- TR 83-08 Formation and properties of actinide colloids U Olofsson B Allard M Bengtsson B Torstenfelt K Andersson Department of Nuclear Chemistry Chalmers University of Technology Göteborg, Sweden 1983-01-30TR 83-09 Complexes of actinides with naturally occurring
- TR 83-09 Complexes of actinides with naturally occurring organic substances - Literature survey U Olofsson B Allard Department of Nucluear Chemistry Chalmers University of Technology Göteborg, Sweden 1983-02-15
- TR 83-10 Radiolysis in nature: Evidence from the Oklo natural reactors David B Curtis Alexander J Gancarz New Mexico, USA February 1983

- TR 83-11 Description of recipient areas related to final storage of unreprocessed spent nuclear fuel Björn Sundblad Ulla Bergström Studsvik Energiteknik AB Nyköping, Sweden 1983-02-07
- TR 83-12 Calculation of activity content and related properties in PWR and BWR fuel using ORIGEN 2 Ove Edlund Studsvik Energiteknik AB Nyköping, Sweden 1983-03-07
- TR 83-13 Sorption and diffusion studies of Cs and I in concrete K Andersson B Torstenfelt B Allard Department of Nuclear Chemistry Chalmers University of Technology Göteborg, Sweden 1983-01-15
- TR 83-14 The complexation of Eu(III) by fulvic acid J A Marinsky State University of New York at Buffalo, Buffalo,NY 1983-03-31
- TR 83-15 Diffusion measurements in crystalline rocks Kristina Skagius Ivars Neretnieks Royal Institute of Technology Stockholm, Sweden 1983-03-11
- TR 83-16 Stability of deep-sited smectite minerals in crystalline rock - chemical aspects Roland Pusch Division of Soil Mechanics, University of Luleå 1983-03-30
- TR 83-17 Analysis of groundwater from deep boreholes in Gideå Sif Laurent Swedish Environmental Research Institute Stockholm, Sweden 1983-03-09
- TR 83-18 Migration experiments in Studsvik
 O Landström
 Studsvik Energiteknik AB
 C-E Klockars
 O Persson
 E-L Tullborg
 S Å Larson
 Swedish Geological
 K Andersson
 B Allard
 B Torstenfelt
 Chalmers University of Technology
 1983-01-31

- TR 83-19 Analysis of groundwater from deep boreholes in Fjällveden Sif Laurent Swedish Environmental Research Institute Stockholm, Sweden 1983-03-29
- TR 83-20 Encapsulation and handling of spent nuclear fuel for final disposal 1 Welded copper canisters 2 Pressed copper canisters (HIPOW) 3 BWR Channels in Concrete B Lönnerberg, ASEA-ATOM H Larker, ASEA L Ageskog, VBB May 1983
- TR 83-21 An analysis of the conditions of gas migration from a low-level radioactive waste repository C Braester Israel Institute of Technology, Haifa, Israel R Thunvik Royal Institute of Technology November 1982
- TR 83-22 Calculated temperature field in and around a repository for spent nuclear fuel Taivo Tarandi, VBB Stockholm, Sweden April 1983
- TR 83-23 Preparation of titanates and zeolites and their uses in radioactive waste management, particularly in the treatment of spent resins Å Hultgren, editor C Airola Studsvik Energiteknik AB S Forberg, Royal Institute of Technology L Fälth, University of Lund May 1983
- TR 83-24 Corrosion resistance of a copper canister for spent nuclear fuel The Swedish Corrosion Research Institute and its reference group Stockholm, Sweden April 1983
- TR 83-25 Feasibility study of EB welding of spent nuclear fuel canisters A Sanderson, T F Szluha, J Turner Welding Institute Cambridge, United Kingdom April 1983
- TR 83-26 The KBS UO₂ leaching program Summary Report 1983-02-01 Ronald Forsyth, Studsvik Energiteknik AB Nyköping, Sweden February 1983
- TR 83-27 Radiation effects on the chemical environment in a radioactive waste repository Trygve Eriksen Royal Institute of Technology, Stockholm Arvid Jacobsson University of Luleå, Luleå Sweden 1983-07-01

- TR 83-28 An analysis of selected parameters for the BIOPATH-program U Bergström A-B Wilkens Studsvik Energiteknik AB Nyköping, Sweden 1983-06-08
- On the environmental impact of a repository for TR 83-29 spent nuclear fuel Otto Brotzen Stockholm, Sweden April 1983
- Encapsulation of spent nuclear fuel -TR 83-30 Safety Analysis ES-konsult AB Stockholm, Sweden April 1983
- Final disposal of spent nuclear fuel -TR 83-31 Standard programme for site investigations Compiled by Ulf Thoregren Swedish Geological April 1983
- Feasibility study of detection of defects in thick TR 83-32 welded copper Tekniska Röntgencentralen AB Stockholm, Sweden April 1983
- The interaction of bentonite and glass with TR 83-33 aqueous media M Mosslehi A Lambrosa J A Marinsky State University of New York Buffalo, NY, USA April 1983
- Radionuclide diffusion and mobilities in compacted TR 83-34 bentonite B Torstenfelt B Allard K Andersson H Kipatsi L Eliasson U Olofsson H Persson Chalmers University of Technology Göteborg, Sweden April 1983 Actinide solution equilibria and solubilities in TR 83-35 geologic systems B Allard Chalmers University of Technology Göteborg, Sweden 1983-04-10 Iron content and reducing capacity of granites and TR 83-36
- bentonite B Torstenfelt B Allard W Johansson T Ittner Chalmers University of Technology

 - Göteborg, Sweden April 1983

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