

Sorption behaviour of well-defined oxidation states

B. Allard U Olofsson B Torstenfelt H Kipatsi

Chalmers University of Technology Göteborg, Sweden 1983-05-15

SVENSK KÄRNBRÄNSLEFÖRSÖRJNING AB / AVDELNING KBS Swedish Nuclear Fuel Supply Co/Division KBS MAILING ADDRESS: SKBF/KBS, Box 5864, S-102 48 Stockholm, Sweden Telephone 08-67 95 40

SORPTION BEHAVIOUR OF ACTINIDES IN WELL-DEFINED OXIDATION STATES

B Allard U Olofsson B Torstenfelt H Kipatsi

Chalmers University of Technology Göteborg, Sweden 1983-05-15

This report concerns a study which was conducted for SKBF/KBS. The conclusions and viewpoints presented in the report are those of the author(s) and do not necessarily coincide with those of the client.

A list of other reports published in this series during 1983 is attached at the end of this report. Information on KBS technical reports from 1977-1978 (TR 121), 1979 (TR 79-28), 1980 (TR 80-26), 1981 (TR 81-17) and 1982 (TR 82-28) is available through SKBF/KBS.

SORPTION BEHAVIOUR OF ACTINIDES IN WELL-DEFINED OXIDATION STATES

B. Allard, U. Olofsson,B. Torstenfelt and H. Kipatsi

Department of Nuclear Chemistry Chalmers University of Technology S-421 96 Göteborg, Sweden

1983-05-15

SUMMARY

The sorption of the actinides Am(III), Th(IV), Np(V), Pa(V), U(VI) and Pu has been studied as a function of pH (2-12) for two nuclide concentrations $(10^{-7}-10^{-9} \text{ M})$ (only one for Pa and U) in the systems Al₂0₃ -0.01 M NaClO₄ and SiO₂ - 0.01 M NaClO₄. Distribution coefficients have been determined by a batch technique after various contact times (6h -6w) at constant temperature (25 $^{\circ}$ C) in systems equilibrated with air. The observed sorption behaviour indicates a predominantly physical adsorption mechanism, where pH of the aqueous phase is the principal chemical parameter of influence. The sorption is highly related to the degree of hydrolysis, with a maximum in the pH-region where neutral species dominate and with a reduction of the sorption under conditions when anionic species (hydroxides or carbonates) would exist in solution This is particularly the case for U(VI) at pH above 7-8 when anionic carbonate complexes would be formed. Plutonium is predominantly tetravalent under the present conditions, as indicated by the sorption behaviour.

CONTENTS

1.	INTRODUCTION		
2.	ACTINIDE REDOX CHEMISTRY AND HYDROLYSIS	1	
	2.1. Oxidation states	1	
	2.2. Hydrolysis and speciation	3	
3.	EXPERIMENTAL		
	3.1. Radionuclides	6	
	3.2. Distribution measurements	6	
4.	RESULTS AND DISCUSSIONS		
5.	CONCLUSIONS		
6.	ACKNOWLEDGEMENTS		
7.	REFERENCES		

ч.

1. INTRODUCTION

The interaction between actinides in environmental waters and exposed solid materials is largely dependent on the chemical state of the element in question (1). Of particular importance are

- o redox conditions; various oxidation states are possible in the environmental pH-range: Th(IV), Pa(V), U(IV)-U(V)-U(VI), Np(IV)-Np(V), Pu(III)-Pu(IV)-Pu(V), Am(III), Cm(III) (2),
- o complexation, especially with hydroxide (hydrolysis) and carbonate

Much of the sometimes contradictory experimental observations concerning the sorption and distribution of actinides in geologic systems can probably be related to poor characterization and control of important chemical parameters such as

- o pH,
- o Eh (redox potential),
- concentration of complexing anions (including carbonate from the water-air exchange),
- o presence of solid agents with non-characterized chemisorbing properties or considerable exchange capacities (pH-dependent).

The purpose of the present study is to get information on the sorption behaviour of actinides under well-defined conditions, using actinides in single oxidation states (Am(III), Th(IV), Np(V), U(VI)) as referenc systems. Hereby it is possible to illustrate the importance of characterizing the redox conditions of e.g. a potential geologic repository site, as well as the effect of changes in Eh or pH on the retention of specific actinides. Some preliminar results from this study have previously been presented (3).

2. ACTINIDE REDOX CHEMISTRY AND HYDROLYSIS

2.1. Oxidation states

The actinides (here denoted by An) exhibit multiple oxidation states (from 3 to 7) in the early part of the series (Th, Pa, U, Np, Pu, Am),

1

but beyond plutonium (Am, Cm, etc.) the lower oxidation states are predominant (2, 4).

Thorium exists only as Th(IV) in aqueous solutions.

For protactinium the most stable oxidation state is Pa(V) in the presence of air, although Pa(IV) can exist in aqueous solutions under highly reducing conditions.

For uranium, U(VI) will dominate entirely in aerated solutions, although both U(IV), and possibly also U(V) would exist under reducing conditions. The lowest oxidation state, U(III), is slowly oxidized by water with the evolution of hydrogen.

For neptunium and plutonium all the oxidation states III, IV, V and VI can exist in aqueous solutions, although Np(III) is easily oxidized. Under oxic conditions Np(V) would dominate entirely, while both Pu(IV) and Pu(V) could exist, as well as Pu(III) (at low pH). The distribution between Pu(IV) and Pu(V) would largely be affected by the carbonate concentration.

Americium is most stable as Am(III), although the easily reduced Am(VI)and Am(V) also can exist in aqueous systems, as well as Am(IV) in the presence of high concentrations of complexing agents. Only Am(III)would be expected in aerated waters in the absence of other oxidants than oxygen from the air.

Curium would largely exist as Cm(III).

Thus, in aqueous solutions equilibrated with air $(Eh \ge 0.8-0.06 \text{ pH})$ and in the pH-range of possible environmental interest (3-4 to 11-12) the following oxidation states would be expected to dominate entirely (1,4-6):

```
An(III):Am
An(IV) :Th
An(V) :Np,Pa
An(VI) :U
```

For plutonium, at least in the intermediate pH-range 5-6 to 8-9 in the absence of any carbonate other than from the carbon dioxide in the

2

air, the expected dominating oxidation state would be Pu(IV), possibly with some minor contribution of Pu(V) at high pH.

2.2. Hydrolysis and speciation

The actinides are generally forming strong complexes with oxygen containing ligands, and the aqueous chemistry is dominated by hydrolysis reactions in the environmental pH-range, as well as by carbonate complexation for species in the penta- and hexavalent state, Table 1.

<u>Table 1</u> Hydrolysis and carbonate complex formation constants (log K_1) for the actinides (An) (25°C, I = 0) (2,6).

		log	K ₁
		L = OH	$L = CO_3^{2-}$
An(III)L	Pu	a	a
	Am	6.5	6
An(IV)L	Th	10.8	
	Pa	14.8 ^c	
	U	13.4	
	Np	12.5	
	Pu	13.5	
An(V)0 ₂ L	Ра	9.5 [°]	
2	U	(4)	(5)
	Np	5.1	5.9
	Pu	4.3	Ъ
An(VI)O_L	U	8.2	10.1
2	Np	8.9	Ъ
	Pu	8.4	(9)

^a Same as for Am assumed

^b Same as for U assumed

^c Ref. 7

The elements exhibit similar behaviour when they are in the same oxida tion state. The differences in complex formation constants for inorganic complexes of environmental interest between the actinides uranium, neptunium, plutonium and americium in the same oxidation state are, for the higher complexes, often within the uncertainty of the data (2). Thorium forms significantly weaker hydroxide complexes than the other tetravalent actinides (size effect). Protactinium differs from the other actinides both in the tetra- and pentavalent state: Pa(V) is more similar to An(IV) than to An(V), in terms of hydrolysis behaviour.

The actinide speciation in aerated solutions are given in Fig. 1. Evidently hydrolysis would become significant at pH above 6 for americium(III), pH l for thorium(IV), pH 7-8 for neptunium(V) and pH 4-5 for uranium(VI). Carbonate complexes would dominate in aerated solutions at pH above 8 for americium(III), pH 8 for neptunium(V) and pH 5-6 for uranium(VI). The speciation in aqueous systems with variation of Eh, pH and $P(CO_2)$ is discussed elsewhere (2,6).



Figure 1 Speciation and distribution coefficients for americium(III), thorium(IV), neptunium(V) and uranium(VI). Distribution data as in Fig. 2-5 (Al₂O₃; Initial concentration: SII (SI for U)(cf. Table 2); 0.01 M NaClO₄).Relative concentration (C_{rel}) calculated from data in ref. <u>2</u>.(Each concentration curve identified by xyzⁿ, corresponding to An_x(OH)_y(CO₃)_zⁿ).



<u>Figure 1</u> Speciation and distribution coefficients for americium(III), Thorium(IV), neptunium(V) and uranium(VI)(Cont.)

3. EXPERIMENTAL

3.1. Radionuclides

Radionuclides according to Table 2 where used in this study. The sorption studies were performed at two different nuclide concentrations for those of the elements where long-lived as well as short-lived isotopes were available. All nuclides were stored in stock solutions of 1-2 M HCl.

Nuclide	Initial concentration ^a		
	SI, Mx10 ⁷	SII, Mx10 ⁹	
²³² Th+ ²³⁴ Th	3.0	2.5	
233 _{Pa}	-	0.004	
233 _U	2.1	-	
$235_{Np}^{237}_{Np}$	1.9	1.9	
²³⁷ Pu+ ²³⁹ Pu	0.6	0.6	
241 _{Am}	2.9	2.3	

Table 2 Radionuclides used in the distribution experiments.

^aConcentration in the solution at the start; the concentration in the stock solutions were about 10^{-5} M, except for ²³³Pa.

The short-lived isotopes 235 Np and 237 Pu were obtained from Harwell, UK, and 234 Th and 233 Pa recovered from 238 U and 237 Np, respectively, by the use of a sorption procedure (<u>8</u>).

The concentrations of ²³³U were measured by liquid scintillation technique (alpha-activity), while all the other nuclide concentrations were determined from measurements of the gamma-activity.

3.2. Distribution measurements

The distribution coefficient $(K_d, mol/kg solid per mol/m³ solution)$ was measured by a batch technique (9,10) under the conditions given in Table 3. The thoroughly sieved and washed solid sorbent and the aqueous phase were contacted, and the active stock solution (typically 0.1 ml per batch of 20 ml) was added. The distribution coefficients were, after an initial pH-adjustment (with NaOH), determined as a function of contact time. No further pH-adjustments were made. Totally 8-12 experiments at various pH but otherwise identical conditions were run simultaneously for each system. The solid and aqueous phases were separated by centrifugation (27000 g for 0.5 h) and samples from the water phases were taken. The distribution coefficients were calculated from

$$K_{d} = [(C_{0}-C)/m]/(C/V)$$
(1)

where C_0 = initial element concentration in solution (mol/1)

C = element concentration after certain contact time (mol/1)

m = mass of sorbent (kg)

V = volume of the water phase (m^3)

No corrections were made for sorption on the vessel walls (made of polypropylene). This should not be required, when considering that the surface areas of the exposed fine-grained solids were several orders of magnitude larger than the surface area of the vessel.

All experiments were performed in aerated systems.

Table 3 Conditions for the distribution measurements.

Solid sorbents:	Cryst. Al_2O_3 and SiO_2 ; grain size 0.090-0.125 mm
Solutions:	0.1 and 0.01 M NaClO ₄ ; variation of pH (2-12)
Radionuclides:	See Table 2
Experimental conditions:	Temperature: 25+1 °C Solid/liquid: 0.20 g/20.0 ml Contact time: 6h, 1d, 1w, 4 or 6w

4. **RESULTS AND DISCUSSIONS**

The measured distribution coefficients (K_d) vs pH for Am(III), Th(IV), Np(V), U(VI), Pa(V) and Pu are given in Fig. 2-7. The curves correspond roughly to a contact time of one week.

Generally, the K_d vs pH-curves exhibit three separate stages:

- increasing K_d with increasing pH; from very low values in the low pH-region to a high level within a fairly narrow pH-range, e.g. from 0.005 to 1.9 (corresponding to 5% and 95% sorption, respectively) within three pH-units for Am(III), Th(IV) and Pu;
- o high and fairly constant K_d in the intermediate pH-region, particularly for Am(III), Th(IV), Pa(V) and Pu;
- o slightly decreasing K_d (Am(III), Th(IV), Pa(V), Pu) or a more pronounced decreasing K_d (U(VI)) with increasing pH in the high pH-region

Only the K_d -increasing stage is observed for Np(V) within the pH-range of the present study (pH<12).



Figure 2 Distribution coefficients for americium(III). O6h, ●27h, □1w, ■6w a. SI, b.-d. SII.

8



<u>Figure 2</u> Distribution coefficients for americium(III)(Cont.). O6h, ●27h, □1w, ■6w.



Figure 3 Distribution coefficients for thorium(IV). O6h, ●27h, □1w, ■6w. a. SI, b,c. SII.



Figure 4 Distribution coefficients for neptunium(V). O6h, ●27h, □1w, ■4w.

a. SI, b. SII.



Figure 5 Distribution coefficients for uranium(VI). O6h, ●27h, □1w, ■6w.



Figure 5 Distribution coefficients for uranium(VI)(Cont.).



Figure 6 Distribution coefficients for protactinium(V). O6h, ●27h, □1w, ■6w.



Figure 7 Distribution coefficients for plutonium. ○6h, ●27h, □1w, ■6w. a. SI, b. SII.

Obviously, the degree of hydrolysis, determined by pH of the system, has a large influence on the distribution between the solid and aqueous phases. The increase in sorption with increasing pH seems to reflect the progressive hydrolysis, as illustrated in Fig. 1, and the sorption maximum roughly corresponds to the pH-range where neutral species would exist. This is particularly the case for the Th(IV)-systems, where no formation of anionic hydroxy species or carbonates would be expected (2,7). The significant decrease in distribution with increasing pH observed for Am(III) and Pu, and the pronounced decrease observed for U(VI) could reflect the formation of anionic species (carbonates and hydroxides). The non-existence of a similar behaviour in the Np(V)-system either indicates a sorption behaviour in this system different from the behaviour of U(VI), or more likely errors in the calculated speciation of Fig. 1 (to high carbonate complex formation constants), as previously pointed out (6).

The general sorption behaviour in the Pu-system strongly indicates that plutonium is predominantly tetravalent under the present conditions (aerated solutions without excess carbonate). Possibly a minor fraction could be either Pu(III) (at pH below 5) or Pu(V) (at pH below 5-6 or above 8-9), considering the general sorption behaviour of Am(III), Th(IV), and Np(V), and the similarity in chemical behaviour of different actinides in the same oxidation state. The predominance of Pu(IV) in the intermediate pH-range with some possible contributions of Pu(III) and Pu(V) is in good agreement with calculated speciation (2, 6).

The hydrolysis dependence of the sorption is further illustrated in Fig. 8, showing pH corresponding to 10% and 50% sorption vs the first hydrolysis constant (log K₁). Here, the assigned log K_d-value might be too high in the plutonium system (Pu(IV) assumed), considering the possible existence of a minor fraction of less hydrolyzed Pu(III) or Pu(V). Considerably lower pH-values for the two selected sorption levels would be expected for Np(V) and U(VI) in the absence of carbonate (carbon dioxide-free atmosphere).

The differences in observed distribution coefficients between the two nuclide concentrations (cf. Table 2) are minor. Slightly higher values are obtained in the high-concentration systems (SI) for thorium and plutonium. However, these systems would be very close to saturation. The solubility product of $An(OH)_4(s)-AnO_2(s)$ would be exceeded in the absence of any sorbing surfaces (2). Slightly higher distribution coefficients were observed in the low-concentration systems (SII) for neptunium. Similar observations have previously been made for uranium (1).

14



Figure 8Hydrolysis dependence of the distribution.Distribution data from Fig. 2-5.0.50% sorption (K = 0.1 m³/kg). $\bullet 10\%$ sorption (K = 0.01 m³/kg).

There is generally a significant increase of the sorption with the contact time, (e.g. from 6h to 1d, 1w and to 4-6w) in the systems where carbonate complexes would not dominate. This can not solely be attributed to the delay due to diffusion into microfissures and crystal lite boundaries in the solid grains (radii around 50 μ m) but rather to slow kinetics in the formation of sorbing higher hydrolysis products or colloidal aggregates. An increased ionic strength appears merely to speed up these processes (11).

In general, three basic kinds of sorption can be distinguished (12):

<u>Physical adsorption</u> processes are due to non-specific forces of attraction between the sorbent and the solute. This type of sorption results in the binding of the solute to solid surfaces in several consecutive layers. This would be a reversible, fairly rapid and largely concentration independent process. <u>Electrostatic adsorption</u> processes (ion exchange) are due to the actior of attractive coulomb forces between charged particles in solution and in the solid. This is often a reversible, fairly rapid and highly concentration dependent process.

The action of chemical forces between a solute and a sorbent would lead to <u>chemisorption</u> processes, e.g. when anions in the solid lattice could form strong complexes with the solute. This would be a highly element specific, partly irreversible, fairly slow and concentration dependent process.

All three of these basic sorption mechanisms can be observed in the interactions of actinides with geologic materials under environmental conditions $(\underline{1})$. However, the sorption mechanism can largely be considered as a physical adsorption process, when

hydrolyzed species or other complexes dominate in the aqueous phase (at pH above 5-6 for Am(III), 3-4 for Th(IV), 7-8 for Np(V), 4-5 for U(VI), 2-3 for Pa(V) and 3-4 for Pu in aerated systems), no chemisorbing agents are present.

The properties of the solid in terms of exchange capacities have only a minor influence on the sorption behaviour under these conditions. (Cf. examples given in ref. <u>1</u> and <u>3</u>: the distribution coefficient for Am(III) in bentonite-systems (cation exchange capacity 750-800 meq/kg) and Al_2O_3 -systems (cation exchange capacity < 1 meq/kg) are very similar at pH 8.5).

5. CONCLUSIONS

The observed actinide sorption behaviour in the present experiments indicate a predominantly physical adsorption mechanism. The major chemical parameters that would have a significant influence on the actinide distribution between a solid and aqueous phase are

- o the redox conditions (determining the oxidation state),
- o pH (determining the degree of hydrolysis),
- o presence of complexing agents (e.g. carbonate).

The sorption of cationic or neutral hydrolyzed actinide species would be pronounced on exposed solid phases, even when the exchange capacity of these solids are low. However, the formation of anionic species (hydroxides, carbonates) would reduce the sorption.

Plutonium is largely tetravalent in aerated aqueous systems of environmental pH in the absence of other complexing agents than the hydroxide as well as the carbonate from the carbon dioxide-water equilibrium.

Further studies of actinide sorption on low-capacity solids in the presence of high concentrations of complexing agents are in progress. Results from these studies, as well as discussions on sorption mechanisms and the various sorption models suggested in the literature will be presented in a subsequent report.

6. ACKNOWLEDGEMENTS

The skilful experimental work by Ms M. Bengtsson, Ms L. Eliasson and Ms W. Johansson is gratefully acknowledged.

7. REFERENCES

- B. Allard, "Sorption of Actinides in Granitic Rock", KBS TR 82-21, Svensk Kärnbränsleförsörjning AB, Stockholm 1982.
- B. Allard, "Actinide Solution Equilibria and Solubilities in Geochemical Systems", KBS TR 83-35, Svensk Kärnbränsleförsörjning AB, Stockholm 1983.
- 3. B. Allard, U. Olofsson, B. Torstenfelt, H. Kipatsi and K. Andersson, "Sorption of Actinides in Well-Defined Oxidation States on Geologic Media", in W. Lutze (Ed.), <u>Scientific Basis for Nuclear Waste Management. Vol. 5</u>, Elsevier Sci. Publ. Comp., New York 1982, p. 775.

- S. Ahrland, J.O. Liljenzin and J. Ryberg, "Solution Chemistry of the Actinides", in <u>Comprehensive Inorganic Chemistry. Vol. 5</u>, Pergamon Press, Oxford 1973.
- B. Allard, "Solubilities of Actinides in Neutral or Basic Solutions", in N. Edelstein (Ed.), <u>Actinides in Perspective</u>, Pergamon Press, New York 1982, p. 553.
- 6. B. Allard, U. Olofsson and B. Torstenfelt, "Environmental Actinide Chemistry", in Proc. 1st Int. Conf. on the Chemistry and Technology of the Lanthanides and Actinides, Venice 1983, in press.
- C.F. Baes and R.F. Mesmer, <u>The Hydrolysis of Cations</u>, John Wiley and Sons, New York 1976.
- 8. B. Allard, U. Olofsson and Y. Albinsson, to be published.
- B. Torstenfelt, K. Andersson and B. Allard, "Sorption of Sr and Cs on Rocks and Minerals. Part I: Sorption in Groundwater", Report Prav 4.29, National Council for Radioactive Waste, Stockholm 1981.
- B. Allard, K. Andersson and B. Torstenfelt, "The Distribution Coefficient Concept and Aspects on Experimental Distribution Studies", KBS TR 83-62, Svensk Kärnbränsleförsörjning AB, Stockholm 1983.
- U. Olofsson, B. Allard, M. Bengtsson, B. Torstenfelt and K. Anders son, "Formation and Properties of Actinide Colloids", KBS TR 83-08, Svensk Kärnbränsleförsörjning AB, Stockholm 1983.
- P. Benes and V. Majer, <u>Trace Chemistry of Aqueous Solutions</u>, Elsevier, Oxford 1980.

1977-78

TR 121 KBS Technical Reports 1 - 120. Summaries. Stockholm, May 1979.

1979

TR 79-28 The KBS Annual Report 1979. KBS Technical Reports 79-01--79-27. Summaries. Stockholm, March 1980.

1980

TR 80-26 The KBS Annual Report 1980. KBS Technical Reports 80-01--80-25. Summaries. Stockholm, March 1981.

1981

TR 81-17 The KBS Annual Report 1981. KBS Technical Reports 81-01--81-16 Summaries. Stockholm, April 1982.

1983

TR 83-01 Radionuclide transport in a single fissure A laboratory study Trygve E Eriksen Department of Nuclear Chemistry The Royal Institute of Technology Stockholm, Sweden 1983-01-19

TR 83-02 The possible effects of alfa and beta radiolysis on the matrix dissolution of spent nuclear fuel I Grenthe I Puigdomènech J Bruno Department of Inorganic Chemistry Royal Institute of Technology Stockholm, Sweden January 1983

- TR 83-03 Smectite alteration Proceedings of a colloquium at State University of New York at Buffalo, May 26-27, 1982 Compiled by Duwayne M Anderson State University of New York at Buffalo February 15, 1983
- TR 83-04 Stability of bentonite gels in crystalline rock -Physical aspects Roland Pusch Division Soil Mechanics, University of Luleå Luleå, Sweden, 1983-02-20
- TR 83-05 Studies in pitting corrosion on archeological bronzes - Copper Åke Bresle Jozef Saers Birgit Arrhenius Archaeological Research Laboratory University of Stockholm Stockholm, Sweden 1983-01-02
- TR 83-06 Investigation of the stress corrosion cracking of pure copper L A Benjamin D Hardie R N Parkins University of Newcastle upon Tyne Department of Metallurgy and Engineering Materials Newcastle upon Tyne, Great Britain, April 1983
- TR 83-07 Sorption of radionuclides on geologic media -A literature survey. I: Fission Products K Andersson B Allard Department of Nuclear Chemistry Chalmers University of Technology Göteborg, Sweden 1983-01-31
- TR 83-08 Formation and properties of actinide colloids
 U Olofsson
 B Allard
 M Bengtsson
 B Torstenfelt
 K Andersson
 Department of Nuclear Chemistry
 Chalmers University of Technology
 Göteborg, Sweden 1983-01-30
 TR 83-09 Complexes of actinides with naturally occurring
- organic substances Literature survey U Olofsson B Allard Department of Nucluear Chemistry Chalmers University of Technology Göteborg, Sweden 1983-02-15
- TR 83-10 Radiolysis in nature: Evidence from the Oklo natural reactors David B Curtis Alexander J Gancarz New Mexico, USA February 1983

- TR 83-11 Description of recipient areas related to final storage of unreprocessed spent nuclear fuel Björn Sundblad Ulla Bergström Studsvik Energiteknik AB Nyköping, Sweden 1983-02-07
- TR 83-12 Calculation of activity content and related properties in PWR and BWR fuel using ORIGEN 2 Ove Edlund Studsvik Energiteknik AB Nyköping, Sweden 1983-03-07
- TR 83-13 Sorption and diffusion studies of Cs and I in concrete K Andersson B Torstenfelt B Allard Department of Nuclear Chemistry Chalmers University of Technology Göteborg, Sweden 1983-01-15
- TR 83-14 The complexation of Eu(III) by fulvic acid J A Marinsky State University of New York at Buffalo, Buffalo,NY 1983-03-31
- TR 83-15 Diffusion measurements in crystalline rocks Kristina Skagius Ivars Neretnieks Royal Institute of Technology Stockholm, Sweden 1983-03-11
- TR 83-16 Stability of deep-sited smectite minerals in crystalline rock - chemical aspects Roland Pusch Division of Soil Mechanics, University of Luleå 1983-03-30
- TR 83-17 Analysis of groundwater from deep boreholes in Gideå Sif Laurent Swedish Environmental Research Institute Stockholm, Sweden 1983-03-09
- TR 83-18 Migration experiments in Studsvik O Landström Studsvik Energiteknik AB C-E Klockars O Persson E-L Tullborg S Å Larson Swedish Geological K Andersson B Allard B Torstenfelt Chalmers University of Technology 1983-01-31

- TR 83-19 Analysis of groundwater from deep boreholes in Fjällveden Sif Laurent Swedish Environmental Research Institute Stockholm, Sweden 1983-03-29
- TR 83-20 Encapsulation and handling of spent nuclear fuel for final disposal l Welded copper canisters 2 Pressed copper canisters (HIPOW) 3 BWR Channels in Concrete B Lönnerberg, ASEA-ATOM H Larker, ASEA L Ageskog, VBB May 1983
- TR 83-21 An analysis of the conditions of gas migration from a low-level radioactive waste repository C Braester Israel Institute of Technology, Haifa, Israel R Thunvik Royal Institute of Technology November 1982
- TR 83-22 Calculated temperature field in and around a repository for spent nuclear fuel Taivo Tarandi, VBB Stockholm, Sweden April 1983
- TR 83-23 Preparation of titanates and zeolites and their uses in radioactive waste management, particularly in the treatment of spent resins Å Hultgren, editor C Airola Studsvik Energiteknik AB S Forberg, Royal Institute of Technology L Fälth, University of Lund May 1983
- TR 83-24 Corrosion resistance of a copper canister for spent nuclear fuel The Swedish Corrosion Research Institute and its reference group Stockholm, Sweden April 1983
- TR 83-25 Feasibility study of EB welding of spent nuclear fuel canisters A Sanderson, T F Szluha, J Turner Welding Institute Cambridge, United Kingdom April 1983
- TR 83-26 The KBS UO₂ leaching program Summary Report 1983-02-01 Ronald Forsyth, Studsvik Energiteknik AB Nyköping, Sweden February 1983
- TR 83-27 Radiation effects on the chemical environment in a radioactive waste repository Trygve Eriksen Royal Institute of Technology, Stockholm Arvid Jacobsson University of Luleå, Luleå Sweden 1983-07-01

- TR 83-28 An analysis of selected parameters for the BIOPATH-program U Bergström A-B Wilkens Studsvik Energiteknik AB Nyköping, Sweden 1983-06-08
- TR 83-29 On the environmental impact of a repository for spent nuclear fuel Otto Brotzen Stockholm, Sweden April 1983
- TR 83-30 Encapsulation of spent nuclear fuel -Safety Analysis ES-konsult AB Stockholm, Sweden April 1983
- TR 83-31 Final disposal of spent nuclear fuel -Standard programme for site investigations Compiled by Ulf Thoregren Swedish Geological April 1983
- TR 83-32 Feasibility study of detection of defects in thick welded copper Tekniska Röntgencentralen AB Stockholm, Sweden April 1983
- TR 83-33 The interaction of bentonite and glass with aqueous media M Mosslehi A Lambrosa J A Marinsky State University of New York Buffalo, NY, USA April 1983
- TR 83-34 Radionuclide diffusion and mobilities in compacted bentonite B Torstenfelt B Allard K Andersson H Kipatsi L Eliasson
 - U Olofsson H Persson Chalmers University of Technology Göteborg, Sweden April 1983
- TR 83-35 Actinide solution equilibria and solubilities in geologic systems B Allard Chalmers University of Technology Göteborg, Sweden 1983-04-10
- TR 83-36 Iron content and reducing capacity of granites and bentonite B Torstenfelt B Allard W Johansson T Ittner Chalmers University of Technology Göteborg, Sweden April 1983

- TR 83-37 Surface migration in sorption processes A Rasmuson I Neretnieks Royal Institute of Technology Stockholm, Sweden March 1983
- TR 83-38 Evaluation of some tracer tests in the granitic rock at Finnsjön L Moreno I Neretnieks Royal Institute of Technology, Stockholm C-E Klockars Swedish Geological, Uppsala April 1983
- TR 83-39 Diffusion in the matrix of granitic rock Field test in the Stripa mine. Part 2 L Birgersson I Neretnieks Royal Institute of Technology Stockholm, Sweden March 1983
- TR 83-40 Redox conditions in groundwaters from Svartboberget, Gideå, Fjällveden and Kamlunge P Wikberg I Grenthe K Axelsen Royal Institute of Technology Stockholm, Sweden 1983-05-10
- TR 83-41 Analysis of groundwater from deep boreholes in Svartboberget Sif Laurent Swedish Environmental Research Institute Stockholm, Sweden 1983-06-10
- TR 83-42 Final disposal of high-level waste and spent nuclear fuel - foreign activities R Gelin Studsvik Energiteknik AB Nyköping, Sweden May 1983
- TR 83-43 Final disposal of spent nuclear fuel geological, hydrological and geophysical methods for site characterization K Ahlbom L Carlsson O Olsson Swedish Geological Sweden May 1983
- TR 83-44 Final disposal of spent nuclear fuel equipment for site characterization K Almén, K Hansson, B-E Johansson, G Nilsson Swedish Geological O Andersson, IPA-Konsult P Wikberg, Royal Institute of Technology H Åhagen, SKBF/KBS May 1983

- TR 83-45 Model calculations of the groundwater flow at Finnsjön, Fjällveden, Gideå and Kamlunge L Carlsson A Winberg Swedish Geological, Göteborg B Grundfelt Kemakta Consultant Company, Stockholm May 1983 TR 83-46 Use of clays as buffers in radioactive repositories Roland Pusch University of Luleå Luleå May 25 1983 Stress/strain/time properties of highly compacted TR 83-47 bentonite Roland Pusch University of Luleå Luleå May 1983 TR 83-48 Model calculations of the migration of radionuclides from a repository for spent nuclear fuel A Bengtsson Kemakta Consultant Company, Stockholm M Magnusson I Neretnieks A Rasmuson Royal Institute of Technology, Stockholm May 1983 Dose and dose commitment calculations from ground-TR 83-49 waterborne radioactive elements released from a repository for spent nuclear fuel U Bergström Studsvik Energiteknik AB Nyköping, Sweden May 1983 Calculation of fluxes through a repository caused TR 83-50 by a local well R Thunvik Royal Institute of Technology Stockholm, Sweden May 1983 GWHRT - A finite element solution to the coupled TR 83-51 ground water flow and heat transport problem in three dimensions B Grundfelt Kemakta Consultant Company Stockholm, Sweden May 1983 Evaluation of the geological, geophysical and TR 83-52 hydrogeological conditions at Fjällveden K Ahlbom L Carlsson L-E Carlsten O Duran N-Å Larsson 0 Olsson Swedish Geological
 - May 1983

TR 83-53 Evaluation of the geological, geophysical and hydrogeological conditions at Gideå K Ahlbom B Albino L Carlsson G Nilsson 0 Olsson L Stenberg H Timje Swedish Geological May 1983 Evaluation of the geological, geophysical and TR 83-54 hydrogeological conditions at Kamlunge K Ahlbom B Albino L Carlsson J Danielsson G Nilsson 0 Olsson S Sehlstedt V Stejskal L Stenberg Swedish Geological May 1983 TR 83-55 Evaluation of the geological, geophysical and hydrogeological conditions at Svartboberget K Ahlbom L Carlsson B Gentzschein A Jämtlid 0 Olsson S Tirén Swedish Geological May 1983 Evaluation of the hydrogeological conditions TR 83-56 I: at Finnsjön Supplementary geophysical investigations of II: the Sternö peninsula B Hesselström L Carlsson G Gidlund Swedish Geological May 1983 Neotectonics in northern Sweden -TR 83-57 geophysical investigations H Henkel K Hult L Eriksson Geological Survey of Sweden L Johansson Swedish Geological May 1983

TR 83-58 Neotectonics in northern Sweden - geological investigations R Lagerbäck F Witschard Geological Survey of Sweden May 1983 Chemistry of deep groundwaters from granitic TR 83-59 bedrock B Allard Chalmers University of Technology S Å Larson E-L Tullborg Swedish Geological P Wikberg Royal Institute of Technology May 1983 TR 83-60 On the solubility of technetium in geochemical systems B Allard B Torstenfelt Chalmers University of Technology Göteborg, Sweden 1983-05-05 TR 83-61 Sorption behaviour of well-defined oxidation states B Allard U Olofsson B Torstenfelt H Kipatsi Chalmers University of Technology Göteborg, Sweden 1983-05-15 TR 83-62 The distribution coefficient concept and aspects on experimental distribution studies B Allard K Andersson B Torstenfelt Chalmers University of Technology Göteborg, Sweden May 1983 Sorption of radionuclides in geologic systems TR 83-63 K Andersson B Torstenfelt B Allard Chalmers University of Technology Göteborg, Sweden 1983-06-15 TR 83-64 Ion exchange capacities and surface areas of some major components and common fracture filling materials of igneous rocks B Allard M Karlsson Chalmers University of Technology E-L Tullborg S Å Larson Swedish Geological Göteborg, Sweden May 1983